



Curriculum Vitae

Sanggeol Jeong

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Ulsan National Institute of Science and Technology
UNIST-Gil 50(44919), Ulsan, Republic of KOREA

AFFILIATION

4th semester at Master Program
Ulsan National Institute of Science and Technology (UNIST)
UNIST-Gil 50, Ulju-gun, Ulsan, 44919, Republic of Korea

EDUCATION

Bachelor of Engineering

- Major: Nuclear Science and Engineering
Energy Conversion and Storage
- UNIST, Ulsan, Korea, 2009.03~2016.02

WORK EXPERIENCE

- Criticality Evaluation on Spent Fuel Transportation Casks and Storage Pools
- In-house Monte Carlo Code Verification
- Generate P1 Sensitivity Coefficient
- Shielding design validation for Sodium-cooled Fast Reactor
- Internship
 - Korea Hydro & Nuclear Power Central Research Institute (KHNP, June 22 ~ August 14, 2015)
 - Undergraduate Research Opportunities Program, University of Illinois at Urbana-Champaign (UIUC, Jan 16 ~ Feb 17, 2016)
 - Organization for Economic Co-operation and Development, The Nuclear Energy Agency (OECD/NEA, Sep 1, 2016 ~ Feb 15, 2017)

RESEARCH INTEREST

- Criticality Safety Analysis on Dry Cask
- Statistical Analysis Methodology on Sub-Criticality Estimation
- Monte Carlo Code Development
- Research Reactor Modelling using MC Code
- P1 Angular Scattering Sensitivity analysis
- Design of Shielding for Sodium-cooled Fast Reactor

CERTIFICATES

- Awards & Scholarship
 - Best Student Paper (M&C 2017, April, 2017)
 - Nuclear Technology Graduate Student Society Scholarship, National Research Foundation of Korea (June, 2016 ~ June, 2017)
 - UNIST Scholarship for outstanding freshman (Mar. 2009)
- TRAINING (International)

- Reactor PPhysics Asia eXperiment Program (α XP) (KUCA, Japan, January, 2018)
- TRAINING (Domestic)
 - Reactor Core Design: Theory and Practice, KAERI (Daejeon, August, 2016)
 - Monte Carlo Theory and MCNP User Training, Hanyang University (Seoul, August, 2017)

**PUBLICATIONS
SCI Journal**

1. Jaerim Jang, Wonkyeong Kim, **Sanggeol Jeong**, Eun Jung, Jinsu Park, Mattieu Lemaire, Hyunsuk Lee, Deokjung Lee*, “Validation of UNIST Monte Carlo Code MCS for Criticality Safety Analysis of PWR Spent Fuel Pool and Storage Cask”, J. Nucl. Sci. Technol., Under review (2017)

**International and
Domestic Conferences**

1. **Sanggeol Jeong**, Wonkyeong Kim, and Deokjung Lee*, “Design of High Density Spent Fuel Storage Rack Applying Burnup Credit,” Transaction of KNS Spring Meeting, Jeju, Republic of Korea, May 16-18 (2018)
2. **Sanggeol Jeong**, Ian Hill, Hiroshi Kikusato, and Deokjung Lee*, “Creation of a Database of Uncertainties for ICSBEP Handbook and Tool for Covariance Generation,” M&C, Jeju, Republic of Korea, April 16-20 (2017)
3. **Sanggeol Jeong**, Jaerim Jang, Wonkyeong Kim, Azamat Khassenov and Deokjung Lee*, “Applications of NUREG/CR-6361 and NUREG/CR-6698 Methodologies to PWR Spent Fuel Transportation Casks and Storage Pools,” Transaction of KNS Spring Meeting, Jeju, Republic of Korea, May 17-19 (2017)
4. Ian Hill, **Sanggeol Jeong**, “STATUS AND ANALYSIS OF P1 ANGULAR SCATTERING SENSITIVITY DATA AVAILABLE WITHIN THE DATABASE FOR ICSBEP (DICE)”, M&C, Jeju, Republic of Korea, April 16-20 (2017)
5. Jaerim Jang, Jinsu Park, Wonkyeong Kim, **Sanggeol Jeong**, Deokjung Lee*, Kyoong-ho Cha, “Validation of UNIST Monte Carlo Code MCS for Criticality Safety Analysis”, KNS Spring Meeting, Jeju, Republic of Korea, May 11-13 (2016)

**ENGLISH
CERTIFICATION**

TOEIC 845
TOEIC Speaking 130

COMPUTER SKILL

Fortran programming, Python script, MATLAB script, Shell script

**REACTOR CORE
ANALYSIS CODE**

MCNP, Serpent, CASMO, SIMULATE, SCALE

COURSES TAKEN

Fundamentals of Nuclear Engineering
Introduction to Nuclear Fuel Cycle Engineering
Nuclear Radiation Engineering &Experiment
Nuclear Materials Engineering &Experiment
Introduction to Nuclear Reactor Theory
Nuclear Engineering Design and Lab I
Nuclear System Engineering &Experiment
Nuclear Reactor Lab
Introduction to Nuclear Engineering IT
Nuclear Reactor Numerical Analysis
Numerical Analysis and Applications (graduate)
Nuclear Reactor Core Analysis (graduate)
Nuclear Dynamics (graduate)